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European Patent Office  
Office européen des brevets

⑪ Publication number:

**0 167 403**

**B1**

⑫

## EUROPEAN PATENT SPECIFICATION

⑯ Date of publication of patent specification: **09.05.90**

⑮ Int. Cl.<sup>5</sup>: **G 21 F 9/34**

⑯ Application number: **85304786.8**

⑯ Date of filing: **04.07.85**

⑯ Land disposal site for the disposal of nuclear waste.

⑯ Priority: **05.07.84 US 627977**

⑯ Date of publication of application:  
**08.01.86 Bulletin 86/02**

⑯ Publication of the grant of the patent:  
**09.05.90 Bulletin 90/19**

⑯ Designated Contracting States:  
**BE CH DE FR GB IT LI NL SE**

⑯ References cited:  
**DE-A-3 410 370**  
**DE-B-1 539 749**  
**GB-A-2 128 801**

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### Description

This invention generally relates to land disposal sites for toxic or nuclear waste materials having a non-structural cap supported by a solidly packed array of waste-containing modules which are arranged to be flexibly conformable with changes in the shape of the site brought about by seismic events or other natural disturbances.

Burial systems for burying nuclear waste are known in the prior art. In the earliest of these systems, such wastes were merely packed into 250 litre (55-gallon) steel drums, dropped into a simple, earthen trench by a long-boom crane, and buried. Unfortunately, such "kick and roll" burial systems proved to be generally unsatisfactory for the land disposal of nuclear waste. The loose soil which these trenches were filled in was much more permeable to water than the densely-packed soil which formed the size of the trench, or the dense rock strata which typically formed the bottom of the trench. Consequently, the relatively loose and water permeable soil which surrounded the drums caused these trenches to collect large amounts of standing water in what is known as the "bathtub effect". This standing water ultimately caused the steel walls of the drums buried within the trenches to corrode and collapse. The collapsing drums and compaction of the soil over time resulted in a downward movement or subsidence of the soil, which caused a depression to form over the top of the trench. This depression in turn collected surface water and hence worsened the tendency of the trench to collect and maintain a pool of standing water over the drums. The resulting increase in standing water resulted in still more soil subsidence and accelerated the corrosion and collapse of the drums buried therein. The corrosion and collapse of the drum containers in such sites resulted in some radioactive contamination of the ground water flowing therethrough.

To solve the soil subsidence and water accumulation problems associated with such "kick and roll" disposal sites, a variety of alternative burial systems have been developed. These alternatives include earthen vaults having structurally rigid walls, and container burial sites which the spaces between the waste containers are filled in with concrete or some other hardenable grout. While these alternative systems constitute clear advances over the trenches used in the simple "kick and roll" disposal systems, various shortcomings are associated with both. For example, where earthen vaults are used which incorporate structurally rigid walls, such rigid walls are apt to crack and break in response to a seismic disturbance. Once the integrity of the vault walls is gone ground water can flow in and accumulate around the waste packages. If any of these packages has metallic walls, the standing water surrounding them can cause the walls to corrode and leach radioactive waste into the ground water. Because such vaults typically have only one access opening, the recoverability of a single, leaking package

would be extremely difficult, if not impossible. While burial sites in which a hardenable substance is poured over a large group of waste containers to form a solid, integral monolith may be more resistant to cracking or breakage due to seismic disturbances, this particular type of disposal site would tend to place very high, localized stresses on the waste containers located in the paths of any faults or cracks which develop in the monolith. Moreover, this type of site has an even worse problem with recoverability when a seismic disturbance does succeed in rupturing only a few or one of the containers encapsulated in the grout. A relocation of the site might be the only solution if such a cracking or breaking of the inaccessible containers occurred.

Clearly, a need exists for a disposal site which is structurally stable, yet flexible conformable to seismic events or other natural disturbances, so that no localized, container-cracking stresses can occur. Additionally, it would be desirable if individual containers buried in such a site were easily and conveniently recoverable in order to obviate relocating the entire site should a seismic event or other natural disturbance break a few or only one of the containers buried therein. Further, the burial site should have means for preventing the accumulation of standing water around the array of containers to insure the longevity of the containers buried therein. Finally, it would be desirable if such a site were easily constructed of inexpensive materials.

The invention consists in a disposal site for the disposal of toxic or radioactive waste material, said site comprising a trench in the earth characterized in that said trench is having a substantially flat bottom lined with a layer of solid, fluent material for obstructing any capillary-type flow of ground water to the interior of the trench; a nonrigid radiation-blocking cap formed from a first layer of alluvium, a second layer of solid-fluent material for blocking any capillary-type flow of water between the layer of alluvium and the rest of the cap, a layer of water-shedding silt sloped toward drain means for directing surface water away from the trench, and a layer of rip-rap over the silt layer for protecting the silt layer from erosion and for providing a radiation barrier, and a solidly-packed array of modules of uniform size and shape disposed in the trench and under the cap for both encapsulating said radioactive wastes from water and for structurally supporting said cap, wherein each module in the array is slidably movable in the vertical direction in order to allow the array of modules to flexibly conform to variations in the shape of the flat trench bottom caused by seismic disturbances, and to facilitate the recoverability of the modules.

The modules used in the disposal site of the invention are preferably in the shape of right-angled prisms which may be solidly stacked into layers which are mutually slidable over one another, as well as columnar stacks wherein each column is slidably movable in the vertical direction relative to the other columns. Such a solidly-

packed arrangement affords a subsidence-free structure which is capable of supporting the non-rigid cap of the site, but which is flexibly conformable to changes in the shape of the earth brought about by seismic or other natural disturbances.

Finally, the invention also encompasses a process for burying nuclear waste material contained within a plurality of uniformly shaped, solidly-packable modules by digging a trench; solidly packing the trench from one to the other with the solidly-packable modules, characterized by placing the non-rigid, water-shedding and radiation-blocking cap formed from fluent, solid material over the tops of the modules as the trench is filled with a solid array of such modules in order to minimize the exposure of the surrounding area to radiation emanating from the modules.

In order that the invention can be more clearly understood, convenient embodiments thereof will now be described, by way of example, with reference to the accompanying drawings in which:

Figure 1 is a perspective, cutaway view of a packaging facility;

Figure 2 is a perspective, cutaway view of the high-force compactor used in the packaging facility illustrated in Figure 1;

Figure 3 is a perspective, cutaway view of a disposal site;

Figure 4A is a top, plan view of the packaging module;

Figure 4B is a side, partial cross-sectional view of this module;

Figure 4C is a bottom view of the module;

Figure 5A is a top view of the cap of the module;

Figure 5B is a side, partial cross-sectional view of the cap illustrated in Figure 5A;

Figure 6 is a perspective view of a packed and sealed module, and

Figure 7 is a perspective, cutaway view of a packed module.

With reference now to Figure 1, wherein like reference numerals designate like components throughout all of the several figures, the packaging facility 1 of the system of the invention generally comprises four isolation walls 2a, 2b, 2c and 2d which enclose a remote handled waste packaging section 3 on the left side of the building, a module loading and transportation section 60 in the center of the building and a contact handled waste section 85 on the right side of the building. Both the remote and contact handled waste sections 3 and 85 include a drive-through 7 and 87, respectively. At these drive-throughs 7 and 87, trucks 13 and 95 deliver remote and contact handled nuclear waste in relatively lightweight shipping containers (i.e., liners 250 litre (55-gallon) drums, and LSA containers) from remotely located waste generating sites for encapsulation into the relatively heavy, solidly packed modules 200. In the preferred embodiment, the final disposal site 150 of the modules 200 packed by the packaging facility 1 is located in close proximity to the facility 1 in order to minimize the distance which the packed modules 200 (which may weigh 13 636 kg (30,000 pounds) must be transported. At the outset, it

should be noted that there are at least three major advantages associated with a facility surrounded by isolation walls which is remotely located from the waste-generating sites, yet is close to a final disposal site 150. First there is no need to transport the relatively heavy modules 200 to the waste generating site. Second, the possibility of the waste-generating site from becoming contaminated from a packaging accident is eliminated. Thirdly, the isolation walls 2a, 2b, 2c and 2d minimize the possibility of the disposal site 150 becoming contaminated from any packaging accidents.

Turning now to a more specific description of the remote-handled waste section 3 of the facility 1, this section 3 includes a driveway 9 having an entrance (not shown) and an exit 11 for receiving a delivery truck 13. Such trucks 13 will normally carry their loads of nuclear waste in a reusable, shielded shipping cask 15 of the type approved by the U.S. Department of Transportation or the U.S. Nuclear Regulatory Commission. Disposed within such shielded shipping casks 15 are metallic or plastic liners (not shown) which actually hold the wastes. Section 3 of the facility 1 further includes a processing platform 18 which is about the same height as the height of the bed of the truck 13, a shield bell 19 having a hook assembly 21, and a remote-controlled traveling crane 23. The shield bell 19 is preferably formed from a steel shell having a lead liner which is thick enough to reduce the amount of radiation emanated from the non-contact waste to an acceptable level. The crane 23 includes a primary hoist 25 detachably connectable to the hook assembly 21 of the shield bell 19 via an electric motor-operated pulley assembly 27. The traveling crane 23 further includes a carriage 29 for moving the primary hoist 25 in the "X" direction (parallel to the driveway 9 of the drive-through 7), as well as a trolley 33 for moving the primary hoist 25 in a "Y" direction (parallel to the front face of the facility 1). The vertically adjustable, electric motor-operated pulley assembly 27, in combination with the carriage 29 and trolley 33, allows the traveling crane 23 to swing the shield bell 19 over the shipping cask 15 of the delivery truck 13, pick up the waste-containing liner out of the cask 15, and place the liner at a desired position onto the processing platform 18. Although a remote-controlled traveling crane 23 operated via a T.V. monitor is used in the preferred embodiment, any number of other types of existing remote-controlled crane mechanisms may be used to implement the invention. In addition to primary hoist 25, a secondary hoist 35 is also connected between the traveling crane 23 and the shield bell 19. The secondary hoist 35 controls the position of a cable and hook (not shown) inside the shield bell 19 which is capable of detachably engaging the waste-containing liner disposed within the shielded shipping cask 15.

The remote-handled waste section 3 of the building 1 further includes a characterization station 37 having various radiation detectors 39 and ultrasonic detectors 41 for verifying that the

contents of the liner inside the shipping cask 15 conform to the shipping manifest. The radiation detectors 39 are used to measure the intensity of the radiation emanating from the waste contained in the liner and to check the "signature" of the radiation spectrum of the waste to confirm the accuracy of the shipping manifest. The ultrasonic detectors 41 are used to determine whether or not any radioactive liquids are present with the liner. Federal regulations strictly prohibit the burial of radioactive wastes in liquid form; consequently, the information provided by the ultrasonic detectors 41 is of paramount importance. Both the radiation detectors 39 and ultrasonic detectors 41 are electrically connected to a bank of read-out dials 45 by means of cables disposed in grooves 43 in the processing platform 18. Although not specifically shown in any of the several figures, the outputs of the radiation detectors 39 and the ultrasonic detectors 42 are preferably fed into a central computer both for record-keeping purposes, and for determining how much of a particular kind of waste can be loaded into a particular module before the surface radiation of the module 200 exceeds a preselected limit. The central computer can further compute how much grout must be poured into a particular loaded module in order to properly encapsulate the wastes, and has the capacity to actuate an alarm circuit when the ultrasonic detectors 41 indicate that an unacceptable percentage of the wastes contained in the liner are in liquid form.

In the preferred embodiment, the height of the processing platform 18 is chosen to correspond approximately with the height of the bed of a trailer truck 13 so that any human operators who may be present on the platform 18 when the lid is removed from the cask 15 will not be exposed to the radiation beaming out of the top of the cask. In operation, the shield bell 19 is lowered into the open cask 15, engages the liner contained therein, and then is swung over the sensors 39 and 41 of the characterization station 37 and quickly lowered to within a few centimeters (inches) of these sensors to minimize any of the exposure of section 3 to any radiation beaming out from the bottom of the shield bell 19 which reflects off of the platform 18. In the preferred embodiment, the processing platform 18 is formed from a solid slab of concrete both for the structural solidarity of the facility 1 as a whole, as well as for shielding purposes. This last purpose will become clearer after the structure and function of the lag storage wells 50 is explained hereinafter. While the characterization station 37 of the preferred embodiment includes only radiation detectors 39 and ultrasonic detectors 41 and other types of detectors (such as remote T.V. monitors for visually identifying the waste) may also be included if desired.

Finally, the remote-handled waste section 3 of the facility 1 includes four lag storage wells 50, as well as a remedial action room 53 formed from shielded walls 54 and accessible through shielded doors 55. Each of the lag storage wells 50 includes

5 a generally cylindrical well topped by a disk-shaped cover. The lag storage wells 50 provide a safe and convenient storage area for nuclear waste shipments in which the characterization station 37 has detected the presence of liquids in excessive quantities or other unacceptable conditions. Additionally, the lag storage wells may be used to temporarily store shipments of remote-handled wastes when the grouting station 118 becomes backed up. The materials and thickness of the disk-shaped cap which tops the wells 50 are chosen so as to reduce the amount of radiation beamed into the working area of section 3 from the remote-handled wastes storable therein to within a safe level. The remedial action room provides a separately contained area within the remote handled section 3 of the facility 1 where broken liners (or liners containing liquids) may be properly repaired or treated without any danger of contaminating the main portion of the remote handled section 3, or the facility 1 at large. As will become more evident hereinafter, the provision of a separately contained room 53 to repair the broken walls of a liner is important because the walls of the liner provide one of the three radiation and water barriers within a module 200 when the liner is grouted within one of these modules. When free liquids are found within the waste liners the remedial action room 53, provides a contained area where the liquid may be mixed with suitable absorbants or other solidification media so as to bring it into a solid form acceptable for burial within the purview of present federal regulations. Under normal circumstances, neither the lag storage wells 50 nor the remedial action room 53 is used to process the remote-handled wastes. Instead, after the characterization tests are completed these wastes are usually remotely hoisted through the labyrinth exit 56 formed by shield walls 57a, 57b which form the back of section 3 and placed into a module 200 on a rail cart 64 en route to the grouting station 118.

45 The module loading and transportation section 60 is centrally located within the facility 1 between the remote handled section 3 and the contact handled section 85. The central location of the module loading and transportation section 60 allows it to conveniently serve both the contact and remote-handled sections 3 and 85 of the facility 1. Generally, the module loading and transportation section 60 includes a conventional traveling crane 62 (which includes all the parts and capacities of previously described traveling crane 23) for loading modules 200 which are stacked outside the building 1 onto rail carts 64. These rail carts 64 are freely movable along a pair of parallel loading rail assemblies 66a and 66b. In order to render the rail carts 64 free-moving, the beds 70a and 70b onto which the tracks 68a and 68b are mounted are slightly inclined so that the carts 64 engaged onto the tracks 68a and 68b of the loading rail assemblies 66a and 66b will freely roll down these tracks by the force of gravity. While not shown in any of the several figures,

each of the loading rail assemblies 66a and 66b includes a plurality of pneumatically-actuated stopping mechanisms for stopping the rail carts 64 at various loading, grouting and capping positions along the loading rail assemblies 66a and 66b. The module loading and transportation section 60 includes a return rail assembly 74 having a bed 78 which is inclined in the opposite direction from the beds 70a and 70b of the loading rail assemblies 66a and 66b. The opposite inclination of the bed 78 of the return rail assembly 74 allows the rail carts to freely roll on the tracks 76 by the force of gravity back to a loading position in section 60 after a grouted and capped module 200 has been removed therefrom. Finally, a shield wall 79 (which is preferably formed from a solid concrete wall at least 30,48 cm (12 inches) thick) is placed between the rail assembly 66a and the return rail assembly 74 in order to shield the contact section 85 from any exposure from the remote-handled wastes contained within the shield bell 19 as they are loaded into one of the modules 200 and grouted. This shield wall 79 generally serves the dual function of allowing a contact-handled waste section 85 to be enclosed within the same facility as the remote handled waste section 3, and allowing the use of a common module loading and transportation section 60 for both the remote and the contact handled sections 3 and 85 of the facility 1. This last advantage avoids the provision of duplicate loading and transportation systems.

Turning now to the contact-handled waste section 85, this section of the facility 1 includes many of the same general components present in the remote-handled section 3. For example, section 85 includes a drivethrough 87 including the same sort of driveway 89, entrance 90 and exit (not shown) previously discussed with respect to drive-through 7. Section 85 also includes a processing platform 93 preferably formed from a solid slab of concrete which rises to approximately the same height as the bed of a truck so as to facilitate the unloading of the packaged wastes from the delivery truck 95. Section 85 also includes a pair of characterization stations 107a and 107b.

Finally, section 85 includes a remedial action room 112 for repairing broken containers, and converting liquid and other improperly packaged wastes into an acceptable solid form for burial.

However, despite these common components with section 3, section 85 includes some other components which are unique in the building 1. For example, a relatively light-duty jib crane 99 having a magnetic or vacuum hoist 101 is used in lieu of the relatively heavy traveling crane 23 of section 3. Because the wastes which are processed in section 85 are of a sufficiently low radiation level so that they may be directly contacted by human workers, there is no need for a crane capable of lifting the heavy shield bell 19 used in section 3. Consequently, the crane used in section 85 need only be capable of lifting lightly-packaged nuclear wastes, which typically arrive at the building 1 in 55-gallon steel drums 97.

Although some light shadow shields may be used on the contactable section 85 of the building 1, the generally low radiation level of the wastes processed in this area obviates the need for heavily shielding each of steel drums 97 containing the wastes. Therefore, a conveyor system 103 preferably formed from rollers is provided which greatly facilitates the handling of the drums 97 in which the wastes are contained. Finally, a high-force compactor 110 is provided which not only compacts the wastes into a smaller volume, but squeezes the surrounding drum down to a point so far above the inelastic limit of the steel that the wastes are incapable of "springing back" in volume during the grouting process. This is an important advantage which will be elaborated on at a later point in this text.

The conveyor system 103 includes both a pair of serially arranged compactor conveyor belts 105a and 105b, as well as a remedial action conveyor belt 106. Compactor conveyor belt 105a conveys the 55-gallon drums 97 containing the contact-handled waste from the jib crane 99 through a first characterization station 107a which includes ultrasonic and radiation detectors (not shown), and into the loading mechanism 110.1 of the high-force compactor 110. The high-force compactor 110 applies a pressure of between 500 and 1,100 tons to the 250 litre (55-gallon) drum containers, thereby reducing them into high-density "pucks" 117 having a density of between 0,9599 to 1,1199 g/cm<sup>3</sup> (60—70 lbs./cu. ft.). In the preferred embodiment, a compaction force of 600 tons is typically used. The high-density pucks 117 are ejected from the high-force compactor 110, and slide down a ramp 111.2 onto compactor conveyor belt 105b, which in turn facilitates the movement of pucks 117 through a second characterization station 107b which is likewise equipped with ultrasonic and radiation detectors (not shown). The conveyor belt 105b then conveys the high-density puck 117 to the magnetic or vacuum hoist 116 of a jib crane 114, which swings the puck 117 over into a module 200 en route to the grouting station 118. The remedial action conveyor belt 106 comes into play when the characterization station 107a detects that (a) the drum 97 contains a liquid, (b) the walls of the drum 97 are broken, or (c) the waste contained within the drum 97 is not compressible. If any of these three conditions are detected, a human operator (not shown) merely pushes the drum 97 from the compactor conveyor 105a onto the remedial action conveyor belt 106, which in turn conveys the drum 97 to the remedial action room 112 where appropriate wall-repairing, liquid solidification, or separate in-drum grouting procedures are undertaken in order to put the drum 97 and its contents in proper condition for encapsulation within a module 200. In the event there is a back-up condition in the remedial action room 112, the drum 97 may be temporarily stored in the lag storage wells 113 of the contact handled section 85.

With specific reference now to Figure 2, the

high-force compactor 110 of the invention includes a loading mechanism 110.1 having a drum scoop 110.2 at the end of an articulated, retractable arm assembly 110.3 as shown. Drums 97 sliding down the chute at the end of the compactor conveyor 105a are fed into the drum scoop 110.2 by a human operator. The articulated, retractable arm assembly 110.3 then loads the drum 97 into a loading cradle 110.4. The compactor 110 further includes a loading ram 110.5 which feeds the drum 97 into a retractable compaction cylinder 110.6 which is movable between a position outside the main ram 110.8, and the top of the ejection ramp 111.2. In Figure 2, the compaction cylinder 110.6 is illustrated in its extended position away from the main ram 110.8, and adjacent the top of the ejection ramp 111.2. After the drum 97 is loaded into the compaction cylinder 110.6, the cylinder 110.6 is retracted into the main ram 110.8, where the drum 97 is crushed between the ram piston 110.9 (not shown), and the bed of the main ram 111.8. As previously mentioned, a compaction force of between 500 and 1,100 tons is applied to the drum 97. There are three distinct advantages associated with the use of such a high compaction force. First, the consequent reduction in volume of the drum 97 and its contents allows many more drums to be packed inside one of the modules 200. Specifically, the use of such a high compaction force allows thirty-five to eighty-four drums 97 to be packaged inside one of the modules 200, instead of fourteen. Secondly, and less apparent, the use of such a high compaction force deforms the steel in the drums 97 as well as the waste contained therein well beyond the inelastic limits of the materials, so that there is no possibility that the resulting, high-density pucks will attempt to "spring back" to a larger shape after they are ejected from the ejection ramp 111.2. The elimination of such "spring back" eliminates the possibility of cavities or internal cracks forming within the hardening grout in the module 200 after the module 200 is loaded with pucks 117 and grouted. Far from "springing back", the resulting high-density pucks 117, when covered with grout, form a positive, non-compressible reinforcing structure in the interior of the module 200 which assists the module in performing its alternative function as a structural support member for the earthen trench cap 164 which is applied over the disposal site 150. Finally, such extreme compaction of the waste inside the drums 97 (which is typically rags, paper and contaminated uniforms) renders them resistant to the absorption of water. This, of course, makes them less prone to leaching out radioactive material in the remote event that they do become wet. Such resistance to water absorption also renders the wastes less prone to biodegradation which again complements the overall function of the module 200 in encapsulating the wastes, since such biodegradation can over time "hollow out" the vessel carrying the waste, and result in subsidence problems.

In closing, it should be noted the compactor 110

includes an air filtration system 111.4 having a filter 111.5, a blower assembly 111.6, and an exhaust stack 111.7. The air filtration system 111.4 draws out any radioactive, airborne particles produced as a result of the application of the 660-1,100 ton force onto the drum 97 carrying the contactable waste.

Turning back to Figure 1, section 85 of the facility 1 includes a grouting station 118 having an extendable trough 120 capable of pouring grout into a module 200 on rail carts 64 engaged to either rail assembly 66a (adjacent the remote-handled waste section 3) or rail assembly 66b (adjacent the contact-handled waste section 85). The use of a single grouting station 118 for modules 200 loaded from both the noncontact and contact handled sections 3 and 85 again avoids the duplication of expensive components in the overall system. Just beyond the grouting station 118 is a capping station 122 including a traveling crane 126 having a hoist 128 for lifting the lids 220 over the tops of the modules 200 incident to the capping process. A more precise description of the capping process will be given when the structure of the modules 200 is related in detail.

While the modules 200 are normally filled with waste, grouted at the grouting station 118 and capped at the waste packaging facility 1 located near the waste disposal site 150, they may also be processed at the facilities of the generator of the waste. Since the surface radiation of the resulting modules is generally low enough for contact handling, the wastes in the modules 200 may be conveniently stored on-site pending the availability of disposal space. When disposal space is available, the modules 200 may be transported in reusable transportation overpacks (not shown) to the disposal site 150 and stacked directly into the trenches 152. While this method is not preferred, it is usually less expensive than using the on-site waste storage facilities.

Figure 3 illustrates the disposal site 150 used in conjunction with the packaging facility 1. The disposal site 150 generally comprises a trench 152 (or a plurality of parallel trenches) having a generally flat, alluvial floor 154. Before the trench is loaded with capped modules 200 in which the grout has hardened, a plurality of water-collecting lysimeters 155 are uniformly placed throughout the floor 154 in order to monitor the radiation level of water in the trench. The lysimeters 155 are placed in the trench floor 154 by augering a hole in the floor, and inserting the elongated bodies of the lysimeters 155 therein. A network of plastic tubes (not shown) enables the operators of the disposal site 150 to periodically draw out any water that has collected in the cups of the lysimeters 155. The radiation level of these water samples is periodically monitored to determine whether or not any radioactive substances have somehow been leached from the modules 200. After the lysimeters 155 have been properly buried throughout the floor 154, the floor 154 is covered with a gravel layer 156 about 61 cm (two

feet) thick, which acts as a capillary barrier. Even though the disposal site 150 is preferably selected in an area where all flow of ground water would be at least 24.38 m (80 feet) below the trench floor 154, the gravel capillary barrier 156 is placed over the top of the floor 154 to provide added insurance against the seepage of ground water into the stacked array 160 of modules 200 by capillary action from the trench floor 154. While all of the capillary barriers in the disposal site 150 of the invention are preferably formed of gravel, it should be noted that the invention encompasses the use of any coarse, granular substance having a high hydraulic conductivity. The layer of gravel 156 is covered with a choked zone of sand 158 approximately four inches thick. This choked zone of sand 158 acts as a road bed for the wheels of the heavy forklifts 185 and trailers 184 which are used to transport the modules 200 to the trench 152. If the zone 158 were not present, the wheels of these vehicles 184, 185 would tend to sink into the gravel layer 156.

The next component of the disposal site 150 is the solidly packed array 160 of hexagonal modules 200 illustrated in Figure 3. In the preferred embodiment, the modules 200 are preferably stacked in mutually abutting columns, with each of the hexagonal faces of each of the modules 200 coplanar with the hexagonal faces of the other two modules forming the column. The arrangement of the modules 200 into such mutually abutting columns results in at least four distinct advantages. First, such solid packing of the modules 200 provides a support structure for the non-rigid trench cap 164 which may be quickly and conveniently formed from natural, fluent substances such as soil, sand and gravel. Second, such an arrangement is almost completely devoid of any gaps between the modules 200 which could result in the previously discussed soil subsidence problems. Third, such an arrangement could weather even severe seismic disturbances, since each of the modules 200 is capable of individual, differential movement along eight different planes (i.e., the top, bottom and six side surfaces of the hexagonal prisms which form the modules 200). Because none of the modules are rigidly interlocked with any of the adjacent modules, each of them is capable of at least some vertical and horizontal sliding movement in the event of a seismic disturbance. Such an eight-plane freedom of movement renders the entire module array 160 flexibly conformable with changes in the shape of the trench 152, and eliminates or at least minimizes the probability of a local seismic disturbance creating local stress points in the array 160 that are powerful enough to rupture or crack the walls of individual containers. Fourthly, the columnar stacking used in the array 160 makes it easy to recover a particular module 200 in the event that such recovery becomes desirable, since any one of the modules 200 may be withdrawn from the trench by digging a single, module-wide hole over the particular column that the desired module is included

within. In the preferred embodiment, the most radioactive or "hottest" of the modules 200 is placed on the bottom layer of the module array 160 and surrounded by less radioactive modules so that the surrounding modules, and the middle and top module layers will provide additional shielding from the radiation emanating from the materials in the "hot" modules.

The trench 152 further includes side gravel capillary barriers 162a and 162b which are positioned between the sides of the solid module array 160, and the walls of the trench 152. Again, the purpose of these barriers 162a and 162b is to prevent any seepage of water from being conducted from the sides of the trench 152 to the sides of the solidly packed array 160 of modules 200. In the preferred embodiment, each of these side capillary barriers 162a and 162b is about 61 cm (two feet) thick.

The trench cap 164 is preferably a non-rigid cap formed from fluent, natural substances such as soil, sand and gravel. Such a cap 164 is more resistant to seismic disturbances than a rigid, synthetic structure would be. Specifically, the non-rigidity of the cap 164 makes it at least partially "self-healing" should any seismic disturbance act to vertically shift the various layers of the cap 164 small distances from one another. Additionally, in the event of a severe seismic disturbance which does succeed in causing considerable damage to the cap 164, the cap 164 may be easily repaired with conventional road building and earth moving equipment. As was previously indicated, the solidly packed array of modules 160 provides all of the structural support needed to construct and maintain the various layers of the trench cap 164.

The first layer of the trench cap 164 is preferably a layer of alluvium 166, which should range from between 122 cm (four feet) thick on the sides to seven 213 cm (seven feet) thick in the center. As is indicated in Figure 3, the alluvium layer 166 (which is preferably formed from the indigenous soil which was removed in creating the trench 152) gradually slips away from the center line of the layer at a grade of approximately 4.5%. Such a contour allows the cap 164 to effectively shed the water which penetrates the outer layers of the cap 164, and to direct this water into side drains 178a and 178b. After the alluvium layer 166 is applied over the top of the solidly packed module array 160, the layer 166 is compacted before the remaining layers are placed over it. Such compaction may be effected either through conventional road bed compacting equipment, or by merely allowing the alluvium in the layer 166 to completely settle by natural forces. Of the two ways in which the alluvium in the layer 166 may be compacted, the use of road bed compaction equipment is preferred. Even though the natural settling time of the alluvium in the invention is very fast as compared to the settling times of soils used in prior art disposal sites, it is still rarely shorter than three months, and may be as long as one year, depending upon the characteristics of

the particular soil forming the alluvium. By contrast, if road compaction equipment is used, the settling time may be reduced to a matter of a few days. It should be noted that the alluvium layer 166 is placed over the solidly packed array 160 at approximately the same rate that the array 160 is formed by stacking the individual modules 200. Such contemporaneous placement of the alluvium layer 166 over the module array 160 minimizes the amount of radiation which the trench workers are exposed to as the disposal site 150 is formed.

After the alluvium layer 166 has been appropriately compacted, a choked zone of sand 168 of approximately 10 cm (four inches) in thickness is applied over it. After the sand layer 168 has been completely applied over the alluvium layer 166, another gravel capillary barrier 170, approximately 61 cm (two feet) in depth, is placed over the choked sand layer 168. The choked sand layer 168 serves as an intrusion barrier between the relatively coarse gravel forming the gravel capillary barrier 170, and the relatively finer alluvium in the alluvium layer 166. Once the gravel capillary barrier 170 has been laid, another choked zone of sand 172, approximately 10 cm (four inches) in thickness, is applied over the gravel capillary barrier 170. Next, a layer of fine, water shedding silt 174 is applied over the choked zone of sand 172 overlying the gravel capillary barrier 170. Again, the choked zone of sand 172 serves as an intrusion barrier between the silt in the silt layer 174, and the gravel in the gravel capillary barrier 170. The silt layer 174 is the principal water-shedding layer of the trench cap 164, and is approximately 61 cm (two feet) thick, and formed from sized material (preferably obtained locally) which is compacted in place. The use of a silt layer 174 in lieu of other water-shedding natural materials, such as clay, is advantageous in at least two respects. First, silt is often more easily obtainable locally than clay, and hence is less expensive. Secondly, if the silt layer 174 should become saturated with water, it will not tend to split or crack when it dries out as clay would. The absence of such splits or cracks helps maintain the overall integrity of the trench cap 164.

The side edges of the silt layer 174 terminate adjacent to the pair of french drains 178a and 178b located on either side of the trench 152. The french drains 178a and 178b include a trench in which perforated pipes 182a and 182b are laid. Water flowing down the sides of the silt layer 174 will float through the perforations in the pipes 182a and 182b and flow along the drain trenches 180a and 180b, away from the trench 152. In the event that the rains or other source of surface water becomes so severe that the silt layer 174 becomes completely saturated with water, the gravel capillary barrier 170 will prevent any water from migrating down from the saturated silt layer 174 into the module array 160 via capillary action.

The top and final layer 176 of the trench cap 164 consists of graded rip-rap which, in more collo-

quial terms, is very coarse gravel (which may be as large as boulder sized). The rip-rap layer 176 performs at least three functions. First, it insulates the silt layer 174 from potentially erosive winds and running water. Second, it provides a final radiation barrier against the module array 160 which brings the radiation level of the disposal site 150 down to well within the range of normal background radiation. Third, it provides an intrusion barrier which discourages would-be human and animal intruders from digging up the ground above the module array 160. The preferred embodiment of the cap 164 as heretofore described is for arid regions. In humid regions, an alternative embodiment of the cap 164 would comprise a first water infiltration barrier of native soil over the solid array 160 of modules 200. This layer in turn would be covered by a sand and gravel capillary barrier similar to the previously discussed layers 168, 170 and 172. These sand and gravel capillary barriers would in turn be covered by a bio-intrusion layer of cobble, and topped by additional sand and gravel layers for supporting a final layer of soil having a vegetative cover. In such an alternative embodiment, the vegetative cover serves both to prevent any erosion which might occur on the upper layer of soil, and also removes water which infiltrates the top layer of the cap. The vegetation used should have shallow roots in order that the integrity of the cap 164 will not be violated. Additionally, such an alternative embodiment might have a steeper slope of perhaps 10° or more because of the greater amount of rainfall associated with such regions.

With reference now to Figures 4A, 4B, 4C and 5A, 5B, the module 200 of the invention generally consists of a container 201 having reinforced concrete walls and a lid 220 which caps the container 201 after it is filled with nuclear wastes and properly grouted.

With specific reference now to Figures 4A through 4C, the container 201 of the module 200 is a hexagonally-shaped prism 202 having a cylindrical interior 216. The corners 204 where the hexagonal walls abut one another are preferably truncated so that small gaps will be left between abutting modules 200 when they are stacked in the module array 160 illustrated in Figure 3. These small spaces are large enough to receive recovery tools (should the recovery of any one of the modules 200 become desirable) but are small enough so that no significant amount of soil subsidence will occur when the modules 200 are arranged in the configuration illustrated in Figure 3. Further the truncated shape of the corners 204 renders these corners less vulnerable to the chipping or cracking which could otherwise occur when the forklift 185 pushes the module 200 into the module array 160 incident to the stacking process.

Turning now to the top and bottom portions of the containers 201 of the modules 200, the top portion 206 is opened as shown to permit the loading of nuclear waste and grout. The top

portion 206 includes three I-bolt anchors 208a, 208b and 208c which allow the container 201 to be handled by the grappling hooks of the cranes in the packaging facility 1 and stacked into the trench 164. Alternatively, these anchors 208a, 208b and 208c allow the modules 200 to be lifted out of the trench 164 if recovery is desired. The shanks of the anchors 208a, 208b and 208c are deeply sunken into the concrete walls of the container 201 as indicated in order to insure an adequate grip thereto. The bottom portion 209 of the container 201 includes the bottom surface 210 of the interior of the container 201, and an outer surface 211 having a pattern of grooves 212. Each of these grooves are slightly deeper and wider than the forks of the shielded forklift 185, so that these grooves 212 greatly facilitate the handling of the module 200 by the forklift 185. The angular pattern of the grooves 212 also allows such a forklift to engage a particular module from a variety of different angles, which further facilitates the handling of the modules. Reinforcing the concrete walls and bottom portion of the container 201 of the module is a "basket" 215 formed from commercially available, steel-reinforcing mesh. The basket 215 greatly increases the tensile strength of the walls and bottom portion 209 of the container 201 of the module 200. In the preferred embodiment, the walls of the container 201 are at least three inches thick. Additionally, the cylindrical interior 216 of the container 201 is at least seventy-five inches in diameter in order that fourteen drums or seven stacks of high-density pucks 117 may be stacked within the cylindrical interior 216 of the container 201. The top portion 206 of the container 201 includes a plurality of grooves 214a, 214b, 214c, 214d, 214e and 214f for receiving the cap-securing rods 232a, 232b, 232c, 232d, 232e and 232f of the slab-type container lid 220, which will be presently discussed in detail.

With reference now to Figures 5A and 5B, the slab-type container lid 220 generally includes a diskshaped upper section 222, and an integrally formed, diskshaped lower section 228 which has a slightly smaller diameter. The edge of the upper section 222 is flattened in three sections 223.1, 223.2 and 223.3, which are spaced approximately 120° from one another. When the container lid 220 is properly placed over the open top portion 206 of the container 201, these flattened sections 223.1, 223.2 and 223.3 should be angularly positioned so that they are, directly opposite the previously discussed I-bolt anchors 208a, 208b and 208c, in order to provide clearance for crane hooks to engage the I-bolt sections of the anchors. The top surface 224 of the upper section 222 of the lid 220 includes a radiation warning symbol 226, which is preferably molded into the face of the lid 220. An identifying serial number may also be molded into the top surface 224 of the lid 220 (as indicated in Figure 3) in order that the module 220 may be easily identified if recovery of the module ever becomes necessary or desirable.

As may best be seen with reference to Figure 5A, three U-shaped transporting lugs 227a, 227c and 227e are placed around the circumference of the upper section 222 of the container lid 220 approximately 120° from one another. These lugs 227a, 227c and 227e are preferably offset from the flattened sections 223.1, 223.2 and 223.3 along the circumference of the upper section 222. Such an angular offset between these lid-transporting lugs 227a, 227c and 227e and the aforementioned flat sections 223.1, 223.2 and 223.3 minimizes the possibility that a crane hook intended for engagement with one of the I-bolt anchors of the module container 201 will inadvertently catch one of the lid transporting lugs 227a, 227c or 227e and accidentally tear it off. As previously mentioned, the container lid 220 further includes an integrally formed lower section 228 which has a slightly smaller diameter than the disk-shaped upper section 222. A layer steel-reinforcing mesh 229 is molded into the concrete forming the container lid 220 in the position shown in Figure 5B. Also molded into the lid 220 are six equidistantly spaced cap-securing rods 232a, 232b, 232c, 232d, 232e and 232f. These rods are slid into the complementary slots 214a, 214b, 214c, 214d, 214e and 214f after the container has been filled with nuclear waste and grouted. Both the container lid 220 and the module container are preferably molded from non-porous portland-based concrete having a compressive tolerance on the order of  $27580 \times 10^3 \text{ N/m}^2$  (4,000 psi). Such concrete is both strong and resistant to penetration by water.

Figures 6 and 7 illustrate a module 200 which has been filled with high-density pucks 117 formed from the high-force compactor 110, and subsequently grouted and capped. In operation, seven stacks of high-density pucks 117 are centrally positioned within the container 201 of the module 200 as shown in Figure 7. The compacted containers which cover the compacted waste form an additional radiation and water barrier between the waste and the exterior of the module 200. Next, the extendable trough 120 of the grouting station 118 of the building 1 pours grout 218 over the seven stacks of pucks 117 so as to form a solid layer of grout between the pucks 117 and the inner surface of the walls of the container 201. In the preferred embodiment, the grout used to fill the module 200 is a  $26875 \times 10^3$  or  $27580 \times 10^3 \text{ N/m}^2$  (3,000 or 4,000 psi) portland-based concrete. However, gypsum, pozzolan, flyash or other cementitious materials may also be used for grout. The hardened grout 218 forms a third radiation and water barrier between the waste in the pucks 117 and the outer surface of the container 200, as is evident from the drawing. The grout 218 also serves to anchor the cap-securing rods 232a, 232b, 232c, 232d, 232e and 232f into the body of the module 200, so that the container 201, the lid 220, the grout 218, and the stacks of pucks 117 become a single, solid structure having a considerable compressive and tensile strength. The completed, hardened modules 200 are carried from the packaging building 1 by drop-

bed trailers 184, and stacked into the solid array 160 illustrated in Figure 3 by means of shielded forklifts 185.

Although not shown in any of the several figures, the module 200 may be specially modified to package special high intensity nuclear wastes such as spent control rods. Specifically, the module 200 may be formed with very thick concrete walls so that a relatively small cylindrical hollow space is left in the center of the module. The control rods may then be transferred directly from a shield transportation cask 15 into the small cylindrical hollow space in the pre-grouted module. Such a modified module may be made longer to accommodate several complete control rods. In the alternative, pre-grouted modules 200 of normal height may be used if the rods are cut up into 11 smaller lengths.

### Claims

1. A disposal site (150) for the disposal of toxic or radioactive waste material, said site comprising a trench (152) in the earth characterized in that said trench is having a substantially flat bottom (154) lined with a layer of solid, fluent material (156) for obstructing any capillary type flow of ground water to the interior of the trench; a non-rigid radiation-blocking cap (164) formed from a first layer of alluvium (166), a second layer of solid-fluent material (170) for blocking any capillary-type flow of water between the layer of alluvium and the rest of the cap, a layer of water-shedding silt (174) sloped toward drain means (178a, 178b) for directing surface water away from the trench, and a layer of rip-rap (176) over the silt layer for protecting the silt layer from erosion and for providing a radiation barrier, and a solidly-packed array (160) of modules (200) of uniform size and shape disposed in the trench and under the cap for both encapsulating said radioactive wastes from water and for structurally supporting said cap, wherein each module in the array is slidably movable in the vertical direction in order to allow the array of modules to flexibly conform to variations in the shape of the flat trench bottom caused by seismic disturbances, and to facilitate the recoverability of the modules.

2. A disposal site according to claim 1, characterized in that said layer (156) of solid, fluent material substantially incapable of conducting water through capillary action is a layer of coarse, granular material having a high hydraulic conductivity.

3. A disposal site according to claim 2, characterized in that the layer of alluvium (166) directly covers said modules.

4. A disposal site according to any of claims 1 to 4, characterized in that the site includes at least one radiation detection device (155) located beneath the floor of the trench for monitoring the radiation level of the trench floor.

5. A disposal site according to claim 1, charac-

terized in that each of the modules (200) is shaped like a right-angled prism having a plurality of flat faces of equal size and shape.

5 6. A disposal site according to claim 5, characterized in that the modules (200) are stacked into a plurality of mutually adjacent columns and wherein the flat faces of each of the modules are arranged mutually coplanar with one another.

10 7. A disposal site according to claim 5, characterized in that each of the modules (200) is a hexagonal prism.

15 8. A process for burying toxic or radioactive waste material contained within a plurality of uniformly shaped, solidly-packable modules by digging a trench; solidly packing the trench from one to the other with the solidly-packable modules, characterized by placing the non-rigid, water-shedding and radiation-blocking cap formed from fluent, solid material over the tops of the modules as the trench is filled with a solid array of such modules in order to minimize the exposure of the surrounding area to radiation emanating from the modules.

20 9. A process according to claim 8, characterized in that the non-rigid, water-shedding and radiation-blocking cap is formed by packing a layer of alluvium over the tops of the modules, allowing the alluvium to settle, and then placing a layer of water-shedding silt over the top of the settled layer of alluvium.

25 10. A process according to claim 9, characterized in that said process further includes the step of placing a layer of coarse, granular material having a high hydraulic conductivity between the layer of settled alluvium and the layer of water-shedding silt in order to provide a capillary barrier between the silt layer and the alluvium layer.

30 11. A process according to claim 10, characterized in that said process further includes the step of covering the layer of silt with a layer of graded rip-rap in order to protect the layer of silt from erosion, as well as to provide a final radiation barrier between the tops of the modules and the surrounding area.

35 12. A process according to claim 11, characterized in that said process further includes the step of monitoring the amount of radioactivity in the water present at the bottom of the trench.

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### Patentansprüche

55 1. Eine Ablagerungsstätte (150) für die Beseitigung von toxischem oder radioaktivem Abfallmaterial, wobei die Stätte einen Graben (152) in der Erde umfaßt, dadurch gekennzeichnet, daß der Graben folgendes aufweist: einen im wesentlichen flachen Boden (154) ausgekleidet, mit einer Schicht von festem, fließfähigem Material (156) zum Unterbinden irgendwelcher kapillarartiger Strömung von Grundwasser zu dem Inneren des Grabens; eine nicht-feste, Radioaktivität blockierende Abdeckung (164), gebildet aus einer ersten Schicht aus Alluvium (166), einer zweiten Schicht von festem, fließfähigem Material (170)

zum blockieren irgendwelcher kapillarartiger Strömung von Wasser zwischen der Schicht aus Alluvium und dem Rest der Abdeckung, eine Schicht aus Wasser abgebenden Trieb sand (174), geformt in Richtung auf Abflußeinrichtungen (178a, 178b), um Oberflächenwasser weg von dem Graben zu führen, und eine Schicht von Geröll (176) über der Triebandschicht zum Schutz der Triebandschicht gegenüber Erosion und zur Lieferung einer Strahlungsbarriere, und eine festgepackte Anordnung (160) von Moduln (200) von gleichförmiger Größe und Gestalt, angeordnet in dem Graben und unter der Abdeckung zur sowohl Einkapselung der radioaktiven Abfälle gegenüber Wasser und zur strukturellen Abstützung der Abdeckung, wobei jedes Modul in der Anordnung in vertikaler Richtung gleitend beweglich ist, um der Anordnung von Moduln zu ermöglichen, sich flexibel an Variationen in der Form des flachen Grabenbodens anzupassen, verursacht durch seismische Störungen, und um die Wiedergewinnung der Moduln zu erleichtern.

2. Eine Ablagerungsstätte nach Anspruch 1, dadurch gekennzeichnet, daß die Schicht (156) aus festem, fließfähigem Material, im wesentlichen unfähig, Wasser mittels Kapillarwirkung zu leiten, eine Schicht von grobem, granulatförmigem Material mit einer hohen hydraulischen Leitfähigkeit ist.

3. Eine Ablagerungsstätte nach Anspruch 2, dadurch gekennzeichnet, daß die Schicht aus Alluvium (166) die Moduln direkt abdeckt.

4. Eine Ablagerungsstätte nach einem der Ansprüche 1 bis 3, dadurch gekennzeichnet, daß die Stätte zumindest eine Strahlungsmeßeinrichtung (155) umfaßt, die unterhalb des Bodens des Grabens angeordnet ist, um den Strahlungspegel des Grabenbodens zu überwachen.

5. Eine Ablagerungsstätte nach Anspruch 1, dadurch gekennzeichnet, daß jedes der Moduln (200) wie ein recht winkliges Prisma gestaltet ist, das eine Vielzahl von ebenen Flächen von gleicher Größe und Form besitzt.

6. Eine Ablagerungsstätte nach Anspruch 5, dadurch gekennzeichnet, daß die Moduln (200) zu einer Vielzahl von aneinander angrenzenden Säulen gestapelt sind, und wobei die ebenen Flächen eines jeden der Moduln gegenseitig koplanar zueinander angeordnet sind.

7. Eine Ablagerungsstätte nach Anspruch 5, dadurch gekennzeichnet, daß jedes der Moduln (200) ein hexagonales Prisma ist.

8. Ein Prozeß zum Vergraben von toxischem oder radioaktivem Abfallmaterial, das in einer Vielzahl von gleichförmig gestalteten, fest verpackbaren Moduln enthalten ist, mittels Ausheben eines Grabens; fest Verpackens des Grabens von einem zum anderen Ende mit den fest verpackbaren Moduln, gekennzeichnet durch Anordnen der nicht-starren, Wasser abgebenden und Strahlung blockierenden Abdeckung, gebildet aus fließfähigem festen Material über den Oberseiten der Moduln, während der Graben mit einer festen Anordnung derartiger Moduln gefüllt wird, um so die Aussetzung des umgebenden Gebietes mit

von den Moduln ausgehender Strahlung zu minimieren.

9. Ein Prozeß nach Anspruch 8, dadurch gekennzeichnet, daß die nicht-starre, Wasser abgebende und Strahlung blockierende Abdeckung geformt wird durch Packen einer Schicht von Alluvium über den Oberseiten der Moduln, Absetzenlassen des Alluviums, und dann Anordnen einer Schicht von Wasser abgebendem Trieb sand über der Oberseite der abgesetzten Schicht von Alluvium.

10. Ein prozeß nach Anspruch 9, dadurch gekennzeichnet, daß der Prozeß weiterhin den Schritt des Anordnens einer Schicht von grobem, granuliertem Material mit einer hohen hydraulischen Durchlässigkeit zwischen der Schicht von abgesetztem Alluvium und der Schicht von Wasser abgebendem Trieb sand umfaßt, um so eine Kapillarbarriere zwischen der Triebandschicht und der Alluviumschicht zu liefern.

11. Ein Prozeß nach Anspruch 10, dadurch gekennzeichnet, daß der Prozeß weiterhin den Schritt des Abdeckens der Schicht von Trieb sand mit einer Schicht von klassiertem Geröll umfaßt die Schicht aus Trieb sand gegenüber Erosion zu schützen, wie auch um eine finale Strahlungsbarriere zwischen den Oberseiten der Moduln und dem umgebenden Gebiet zu liefern.

12. Ein prozeß nach Anspruch 11, dadurch gekennzeichnet, daß der prozeß weiterhin den Schritt des Überwachens der Menge der Radioaktivität in dem Wasser umfaßt, das am Boden des Grabens vorhanden ist.

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### Revendications

1. Site de stockage (150) pour le stockage définitif de déchets toxiques ou radioactifs, ce site comprenant une tranchée (152) creusée dans le sol, caractérisé en ce que la tranchée comporte: un fond (154) sensiblement plat, recouvert d'une couche (156) d'une matière solide, pouvant couler facilement, pour faire obstacle à tout écoulement du type capillaire des eaux souterraines vers l'intérieur de la tranchée; une chape non rigide (164) bloquant les radiations et formée d'une première couche (166) d'alluvions, d'une seconde couche (170) de matière solide, pouvant couler facilement, pour bloquer tout écoulement du type capillaire de l'eau entre la couche d'alluvions et le reste de la chape, d'une couche (174) de limon de déviation d'eau, s'étendant en pente en direction de moyens d'évacuation (178a, 178b) destinés à diriger l'eau de surface loin de la tranchée, et d'une couche (176) d'enrochement, disposée au-dessus de la couche de limon pour protéger la couche de limon contre l'érosion et pour constituer une barrière contre les radiations; et un ensemble étroitement serré (160) de modules (200) de même taille et de même forme, disposés das la tranchée et sous la chape, tant pour emprisonner les déchets radioactifs que pour les protéger contre l'eau ainsi que pour supporter structuralement la chape, chaque module de l'en-

semble de modules pouvant se déplacer par glissement dans la direction verticale afin de permettre à l'ensemble de modules de s'adapter de façon souple aux variations de la forme du fond plat de la tranchée provoquées par des secousses sismiques et pour faciliter la récupération des modules.

2. Site de stockage suivant la revendication 1, caractérisé en ce que la couche (156) de matière solide pouvant couler, sensiblement inapte à conduire l'eau par action capillaire, est une couche de matière granulaire à grains grossiers ayant une conductibilité hydraulique élevée.

3. Site de stockage suivant la revendication 2, caractérisé en ce que la couche (166) d'alluvions recouvre directement les modules.

4. Site de stockage suivant l'une quelconque des revendications 1 à 4, caractérisé en ce que le site comprend au moins un dispositif (155) de détection de radiations situé en dessous du fond de la tranchée pour surveiller le niveau de radiations du fond de la tranchée.

5. Site de stockage suivant la revendication 1, caractérisé en ce que chacun des modules (200) a la forme d'un prisme droit comportant une pluralité de faces plates de mêmes dimensions et de même forme.

6. Site de stockage suivant la revendication 5, caractérisé en ce que les modules (200) sont empilés en plusieurs colonnes mutuellement adjacentes, les faces plates de chacun des modules étant disposées mutuellement dans le même plan les unes avec les autres.

7. Site de stockage suivant la revendication 5, caractérisé en ce que chacun des modules (200) est un prisme hexagonal.

8. Procédé pour enfouir des déchets toxiques ou radioactifs contenus dans une multiplicité de modules, de même forme et pouvant être étroitement groupés, par creusement d'une tranchée,

remplissage de façon serrée de la tranchée d'un côté à l'autre avec les modules pouvant être étroitement groupés, caractérisé par le fait que l'on place une chape non rigide de déviation d'eau et de blocage de radiations, qui est formée d'une matière solide, pouvant couler facilement, au-dessus des modules au fur et à mesure que la tranchée est remplie avec un ensemble compact de ces modules afin de minimiser l'exposition de la zone environnante aux radiations émanant des modules.

9. Procédé selon la revendication 8, caractérisé en ce que l'on forme la chape non rigide, de déviation d'eau et de blocage de radiations, en tassant une couche d'alluvions au-dessus des modules, en laissant les alluvions s'agglomérer, puis en plaçant une couche de limon de déviation d'eau au-dessus de la couche agglomérée d'alluvions.

10. Procédé selon la revendication 9, caractérisé en ce qu'il comprend, en outre, l'étape consistant à placer une couche de matériaux granulaires à grains grossiers ayant une conductibilité hydraulique élevée entre la couche d'alluvions agglomérées et la couche de limon de déviation d'eau afin de former une barrière anticapillaire entre la couche de limon et la couche d'alluvions.

11. Procédé selon la revendication 10, caractérisé en ce qu'il comprend, en outre, l'étape consistant à recouvrir la couche de limon d'une couche d'enrochement à grosseur de grains appropriée pour protéger la couche de limon contre l'érosion ainsi que pour former une barrière finale contre les radiations entre le haut des modules et la zone environnantes.

12. Procédé selon la revendication 11, caractérisé en ce qu'il comprend, en outre, l'étape consistant à surveiller le degré de radioactivité de l'eau présente à la base de la tranchée.

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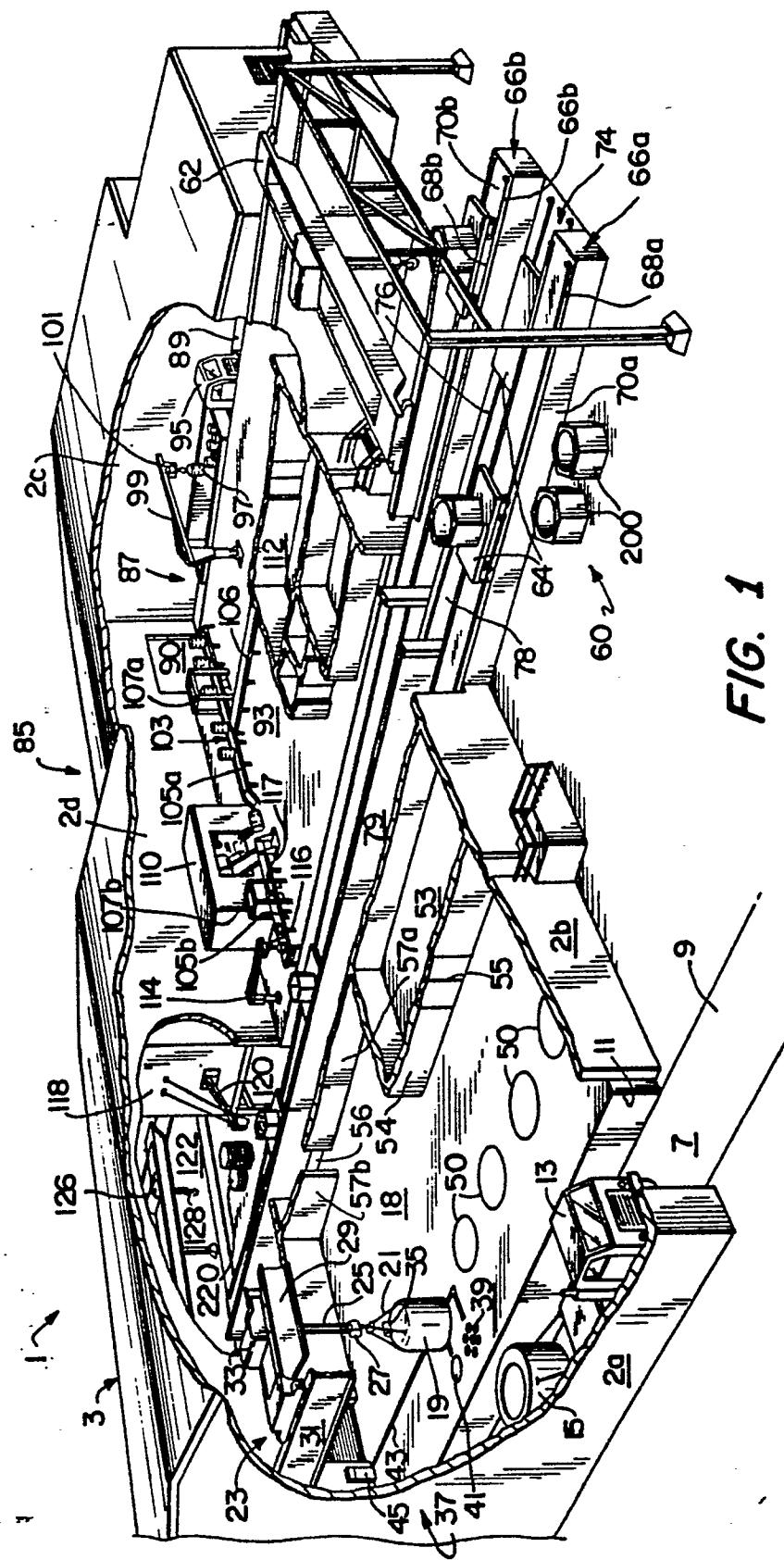
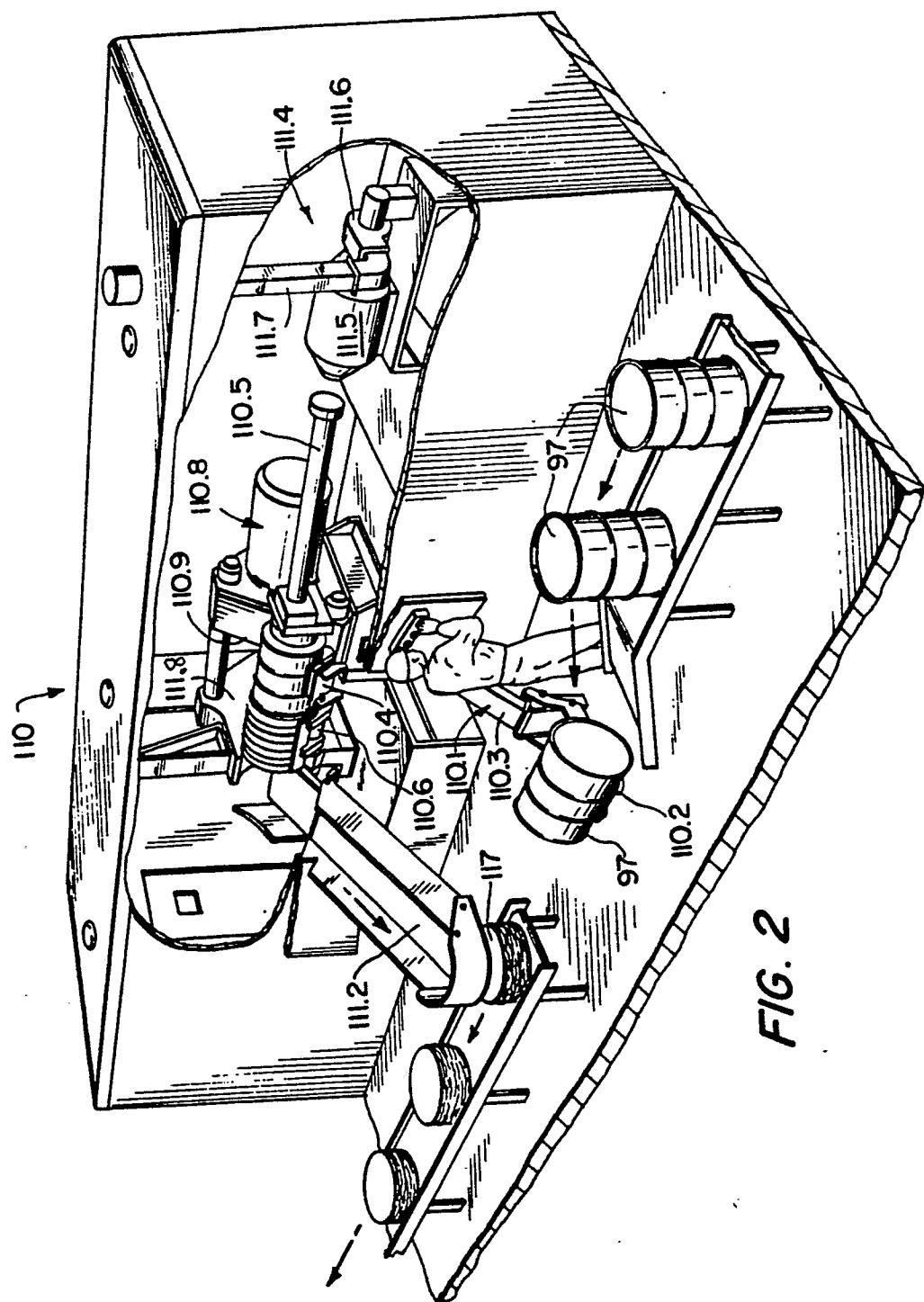
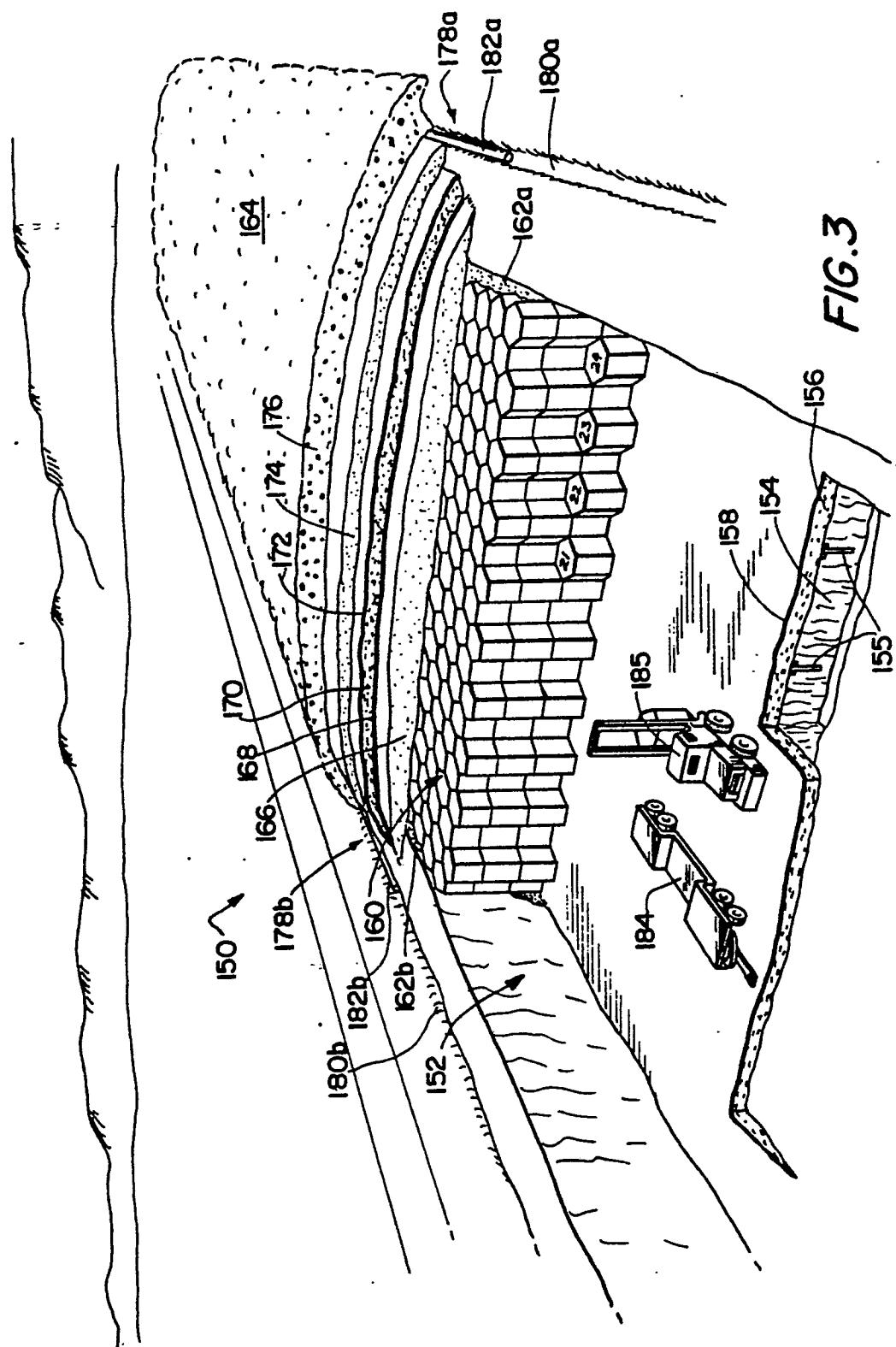


FIG. 1





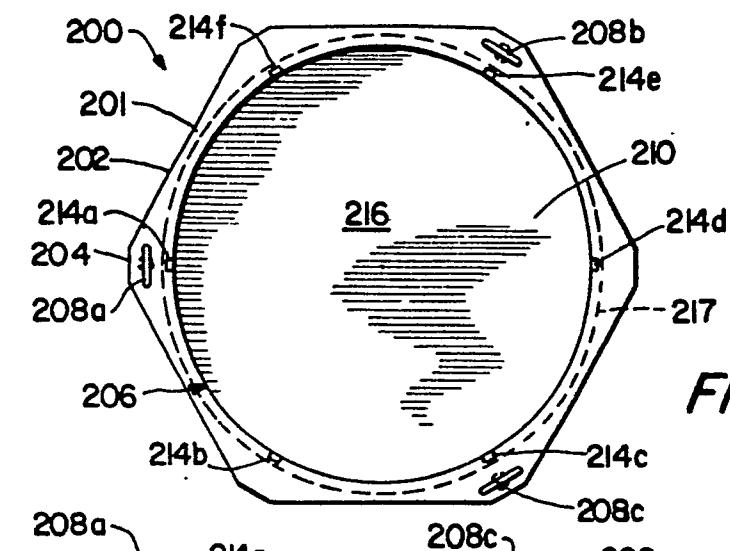


FIG. 4A

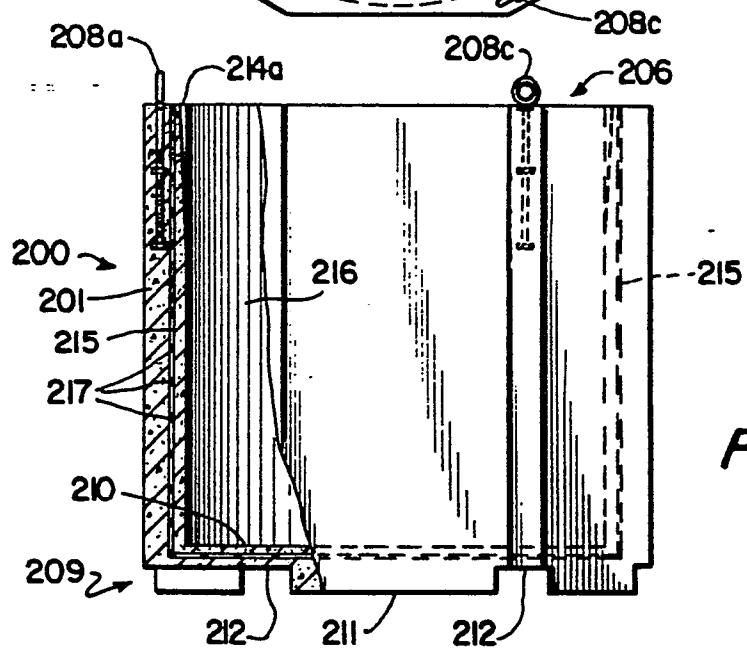


FIG. 4B

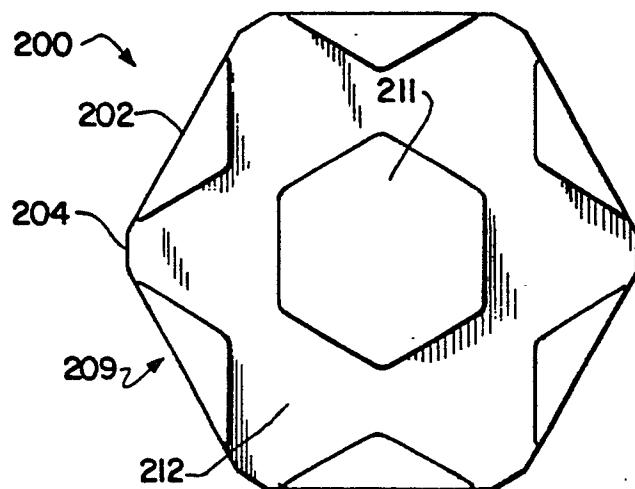


FIG. 4C

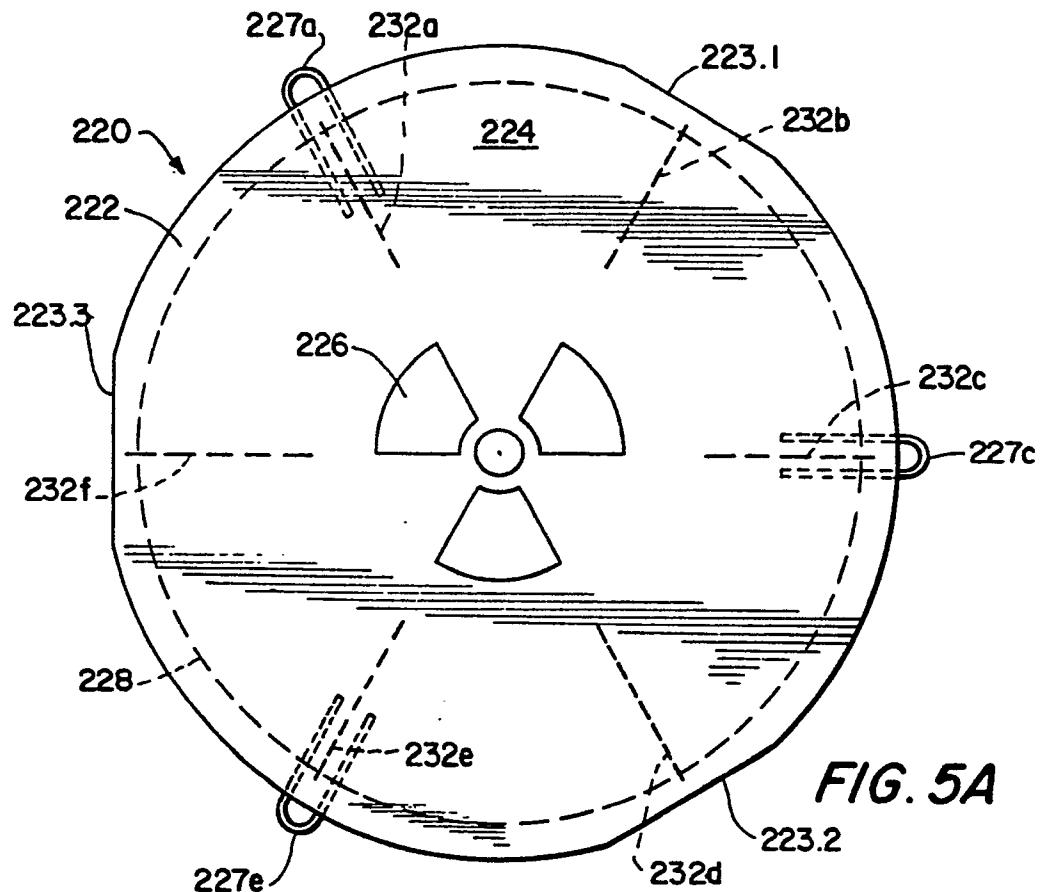


FIG. 5A

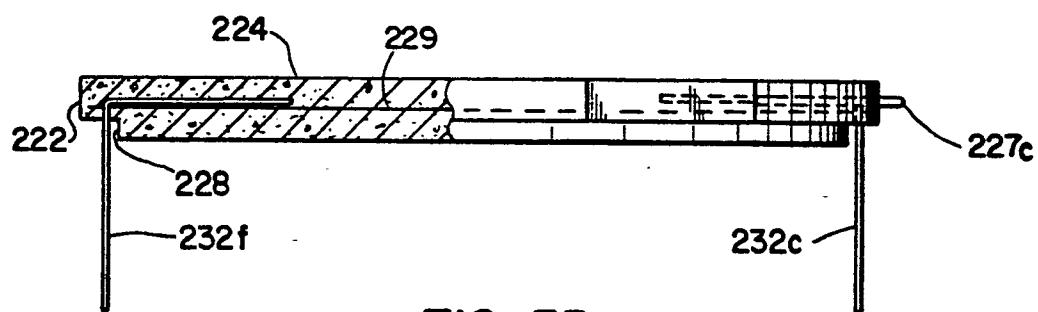


FIG. 5B

